# DIII-D Indicative Time Line to 2030

**Presented by** 

# **RJ Buttery** with thanks to many DIII-D colleagues

For distribution to participants in DOE open call process April 2025







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## **Context of DIII-D Planning**

- DIII-D research team drafted a five year plan for 2024-29
  - From this plan DOE identified a number of projects to cost, also adding new wall

#### • Several of these have been authorized & started/completed funding:

- ECH to 10 lines Chimney divertor Tungsten wall LHCD completion
- Plus various parallel project that may come to DIII-D: pellets, SPF, NB RF sources
- Other projects have not been authorized yet, so are speculative
  - The attached chart shows where they might land if they are funded
  - It is too early to speculate whether FES would be motivated to fund them
- The detailed timeline does evolve as year to year budget decisions, and project designs and execution plans are developed
  - The attached chart shows a realistic integrated plan that includes started projects and options under consideration with FES



Dynamic program environment, responding to need & developments

## Indicative Timeline of DIII-D Facility Development to 2030

CY:	2024 F	2025	F Y	2026 <sup>F</sup> <sub>Y</sub>	20	027 F	2028 <sup>F</sup> <sub>Y</sub>	202	7 F Y	2030 F
		Ops		Ops 16 wks O	ps 16 wks			Vent		Ops
Power	♦ 3MW EC	→ r	ising t	o <b>→ </b>	VIW EC			•	Upper	Divertor Re-opt
	♦ 16MW NB						NB RF sources	→ 20MW		
Exhaust	Shape Rise	Divertor		<ul> <li>'Chimney' D</li> </ul>	ivertor		<ul> <li>Wall change w rovised W dive</li> </ul>	ith 🔶	Add'l v	wall elements
		<ul> <li>Coupor</li> </ul>	n & tile t	esting of new ma	aterials		Teviseu w uwe		Lower I	iv Material 'B'
Innovation	♦ Helicon	+ HFS-L	HCD	• N	NT Armor II	option high	er κ & β 💦 🔶 NT D	ivertor 🔶	Spin Po	l Fusion
mnovalion		•	ITER &	innovative tile te	sting camp	aign	DMS: gas gun, El	A launch 🔶	Runawa	y Electron Coil

Grey=funding tbd

- Projects in black are underway and receiving funding
- Projects in grey are subject to later decisions in DOE's annual funding process for DIII-D
   Actual funding commitments are not usually received until close to project start
- For detailed research plans, see the DIII-D Five Year Plan and the DIII-D Wall Proposal
  - TRL approach implemented to direct and assess progress



Heating, wall & divertor transform DIII-D capabilities to close gaps on FPP

## **Other Notes**

- Note timing of wall vent depends on an engineering plan under development, and will be commenced at the earliest opportunity
  - Possibly a little earlier or later than shown
- DOE call instructs proposers to not propose experiments beyond FY28 on DIII-D
  - This does not preclude ongoing analysis to reach key deliverables and insights
- A separate memo is released on "Essential Service Roles"
  - See link: https://d3dfusion.org/wp-content/uploads/Essential-Service-Roles-at-DIII-D-250408.pdf
  - This covers activities that are essential to facility operation and provision of basic data to characterize plasma for most users.



## For more details of the DIII-D mission

#### • See:

- Physics of Plasmas paper: "DIII-D's role as a national user facility in enabling the commercialization of fusion energy"
  - Phys. Plasmas 30, 120603 (2023), https://doi.org/10.1063/5.0176729
- DIII-D Five Year Plan (for users with internal access):
  - https://fusionga.sharepoint.com/:b:/r/sites/DIII-DHub/Shared%20Documents/Program%20Plans/Five-Year%20Research%20Plans/D3D-FYRP-2024-2029.pdf?csf=1&web=1&e=vP4cfE
- DIII-D Mission for new wall: "Gaps and Alternatives for the First Wall Material in DIII-D" (for users with internal access)
  - https://fusionaa.sharepoint.com/:b:/r/sites/FullWallChanaeOut/Shared%20Documents/4.%20Physics%20Evaluation%20/Gaps%20PVR%20etc.1/DIII-D%20Report%2006%20Gaps%20and%20Alternatives%20fo%20Wall%20Chanae%20-%20release%202410116.pdf?csf=1&web=1&e=livBKk

 Or contact and discuss with DIII-D team members through the usual Record of Discussion process

- Contact form and instructions here: https://d3dfusion.org/become-a-user/#rod

Following slides summarize key capabilities and mission



# Additional Slides on DIII-D's Research Mission

# & Capabilities Enabled by Planned Enhancements



# To Resolve Approach Must Access Right Regime, Innovate & Project



**Controlling variable** 



Through this approach, DIII-D continues to play a critical & needed role in defining future facilities

## DIII-D is a Uniquely Flexible Tool to Resolve Required Solutions



HFS

LHCD

#### High flexibility to discover new solutions

- Shape, 3D fields, fueling, impurities, density control
- Drive/balance rotation, current & heat to e<sup>-</sup>'s or ions
- Rapid change outs to test new technologies, materials and systems in relevant regimes

## Comprehensive measurements → Science

- Over 50 techniques: Kinetics, magnetic, particles, fast ions, neutrals, heat, impurities,
- Profiles, 2D, 3D, and imaging

## Collaboratively led with 700 users

- -21 fields led by uni's, Nat Labs, intl & GA
- Joint development of strategy
- Oversite by independent User Board & PAC

## Supporting ~100 institutions to pursue <u>their</u> priorities with an effective user model





# Fusion Requires an Integrated Core-Edge-Technology Solution

#### Performance optimizes down two paths

- Steady state: Exploit natural improvements in stability
   & transport through shaping, profiles & high β
  - > Lower current, self-driven solutions decrease loads and can be sustained noninductively
  - > Need to validate projected solutions
- Pulsed: High confinement through high current
  - > Robust performance but increased instability, heat & stress
  - > Can stability be maintained?
- Common research needs to address power handling, transients, control, and required technologies

- Resolve compatibility between different parts of solution



Our goal is to explore these challenges and discover new & better solutions





# Address 'Integrated Tokamak Exhaust & Performance' (ITEP) Gap

## Tension between:

- High density radiative divertor solution
- High temperature high performance core

## • Present devices tend to work between these regions

-To overcome must do both

## DIII-D pursuing by

- Shape, volume and current rise
- Heating & current drive rises

high pressure

- Advanced divertor & core configurations with relevant wall
  - > Relevant physics regime for core-edge resolution & better solutions

## Basis to develop integrated solution





# DIII-D Enhancement in Next 5 Years will Confront the ITEP Gap and Resolve Integration with Key Fusion Technologies

#### Increased shaping, volume & current

- Raise pressure & density ightarrow close gap on reactor regimes
- Increase heating and current drive
  - Support high performance dissipative regimes
- Chimney divertor
  - Isolate key physics & test better concept
- Tungsten wall
  - Carbon free to explore new materials & qualify solutions
- Negative triangularity divertor (funding not yet determined
  - Potentially transformational path













Enables program of advanced plasma scenario & technology testing, and their integration

## New Shape Volume & Current Rise Divertor Raises Pressure, Density and Opacity to Confront Core-Edge Challenge



- Increased shaping opens large expanse in operational space
  - Raises pressure and density access
  - Increases opacity & lowers neutral penetration
    - Gradients become transport-defined, like FPP, rather than by neutral deposition

## Increases scope of pedestal exploration



- More advanced pedestals: Scope limits of performance
  - & dissipation through shaping & control techniques

Unique basis for core-edge integration & resolving reactor pedestal science



rise

Increased Heating & Current Drive Supports High Density and Temperature for Core-Edge-Wall Integration





7MW ECH ordered: directable electron heating or current drive, without fueling or torque

> 20MW NBI with RF sources being developed by NC State: bulk heating, on/off axis current drive & co/ctr for rotation control





New helicon current drive: installed & testing



New HFS LHCD installed: testing in 2025



Provides high flexibility & developing new technology

# New Heating and Current Drive Enables DIII-D to Explore Candidate Power Plant Core Solutions

# Core

Current

Density

Broad

## Spectrum of plasma regimes

 From broad → peaked currents, high bootstrap → driven currents

# Heating upgrades provide scope to explore solutions & address physics



Regime	Strength	Challenge			
Broad	βN=5 potential; Low disruptivity	Fast ion transport wall modes			
Hybrid	Efficient CD, Robustness	Current evolution βN limit			
Peaked	Good confine't no RWM	Sustainment; Tearing. Disrupts			

Performance (β) Wall mode kinetic damping and fast ion instabilities vs. current profile

 Burning Plasma Conditions (Ω T<sub>c</sub>/T<sub>i</sub> P<sub>ci</sub>)

 Turbulent transport & kinetic effects in thermalized plasmas at low rotation

thermalized plasmas at <u>low rotation</u> Core-Edge Integration (n, q)

High density and power to understand impurity and core-edge optimization

# rofile H&CD tools: Disrupts Off axis beam ECCD

Peaked

Hybrid



## Unique flexibility to develop scenarios & resolve predictive science for FPP core

# Resonant 3D field ELM suppression with flexible coil arrays QH and other benign ELM regimes: resolve controlling edge

 QH and other benigh LLM regimes: resolve controlling edge physics & ExB rotation requirements with flexible profile control

'peeling limited' pedestals to resolve integrated scenarios

• ELM control: Unique access to relevant low rotation & collisionality

- Pellet pacing: sufficient triggering and heat reduction
- **Plasma control:** Unique headroom through α-like electron heating, with precise deposition & profile control
  - Burn simulation & control with FPP-like actuator and measurement constraints
  - Tearing mode control via direct island deposition or profile control
  - Disruption avoidance: Machine learning, faster-than-RT simulation, sensing
    - Digital twin develops robust schemes offline for testing online

## DIII-D the key proving ground to resolve tokamak control & non-linear multiscale physics of MHD phenomena

# Electron Heating Rise Provides Crucial Capability to Resolve Transient Control in Relevant Regimes







# These Capabilities Enable DIII-D to Address Plasma Behavior and Interaction Questions Across the Board for FPP

E	C							
Lines	Power	←KEY TECHNIQUES →						
6	3-4 MW	Disruption mitigators	Entry point for high q <sub>min</sub> AT	Divertor science & geometry tests	Novel RF technologies	FPP Diagnostic	cs	
		Perturbative transport in H-mode	Shape rise & pedestal density & pressure limits	Radiative techniques	Peeling limited pedestals for ELMs	Materials erosion & transport	Sample &	
8 5.6 MW		ITER dual			testing			
		control, Q=10	AT stability limits	Pulsed FPP scenarios	ELM mitigation low rotation 8	nat ( v*	Control impurity accumulation	
10	7 84/8/	FPP-like fast ions	Alternate ITER	Burn	Divertor science	in	WITH ECH	
10	/ //////	ITER ramp up	scenarios	simulation	opaque conditio	ons Co	Components & materials at high	
+ add	itional NBI,	& steady state	Opaque High performance		ince Mate		, density, q	
Possi	bly more	low rotation, T <sub>e</sub> ~T <sub>i</sub> , high β	collisionless pedestals	& high dissipa core-divertor so with high SOI	Ition integr Iutions with L v*	ation core		
some	eaea ror missions		COR	E – EDGE INTE	GRATION			

## Close plasma research and FM&T gaps for FPP

RJ Buttery/Open Call Background 2025/16

# New "Chimney" Divertor Concept will Resolve Key Physics & May Offer Improved Divertor Solution

## Longer leg

- Isolates physics for model validation
- Avoids X point degradation

## "Chimney" design improves detachment

Mid-leg pump stabilizes radiation front at duct



Divertor



#### SOLPS predicts cold dense target & hot X with good stability







## Test key principles behind divertor design

RJ Buttery/Open Call Background 2025/17

# New Tungsten Wall to Resolve Fusion Solutions in Reactor Relevant Conditions



- Wall a key constraint on the plasma solution
  - Must tolerate core scenario
  - Influence detachment, pedestal, core performance & stability
- DIII-D carbon wall influences core radiation, outgassing & erosion —Time to confront this → DIII-D moving to W wall in 2027
- Adapt DIII-D developed scenarios for W environment,
  - Benefiting from key mitigations in core, pedestal & divertor
- Test innovative new materials without carbon
  - Better solutions needed than tungsten
- Resolve integrated core-edge-wall-technology solutions

Tungsten wall transforms the context of much of DIII-D's research



# FPP Technology Development Program Pioneers New Solutions

HES-LHCE

RJ Buttery/Open Call Background 2025/19

### DIII-D brings key characteristics necessary

- Flexibility, diagnosis, relevant regimes, integration
- Swap out components rapidly & often
  - Difficult  $\rightarrow$  impossible in activated or tritiated devices
- Assess with relevant solutions for wall divertor & core

## Technology Group spans 1/3rd of DIII-D program

- Platform approach with rapid facilitated access
  - Materials, control, diagnostics, components

## Pursue key innovative techniques

- Disruption mitigation: pellets & passive coil
- Helicon & HFS-LHCD RF
- Reactor fueling
- Spin polarized fusion



## Key capabilities that will qualify critical fus

#### Proven track record



#### **Materials interactions**

- Explore degradation
- Understand transport
- Assess divertor leakage

Studies of W & ELM behavior, and new materials



## DIII-D Providing Key Testing Ground for Innovative Materials

- Divertor Materials Evaluation Station

  - well-diagnosed shot-to-shot replacement
    - varving geometries relevant plasma loads

#### Tiles & rings to assess materials on bulk scale





Diagnostic	Interest
IR imaging	Heat flux
Spectrometer	Erosion
Langmuir Thomson	Plasma parameters
Thermocouples	Temperature



Exploring new alloys, ceramics & liquid metals





#### Unique insights and tests possible

RJ Buttery/Open Call Background 2025/20

# Negative Triangularity Provides Transformational Potential for Fusion (not yet funded)

- Negative Triangularity gave high confinement with low power to divertor and no ELMs
  - DIII-D changed hardware to test diverted 'NT'
    - in just two weeks!
  - Exciting results with great confinement & stability
- New closed pumped NT divertor will combine with ECH upgrade to close remaining gaps
  - Core-edge integration:
    - Detachment with high performance core
  - Assess Advanced Tokamak & wall compatibility

# Negative Triangularity could upend the tokamak concept !









Cryo-pumped full closed NT divertor







## Rapid, free, flexible-scope access in as little as a day

# DIII-D Tests Common Research Needs of Novel Fusion Concepts, Fundamental Science and Technology

- Many common fundamental processes behind fusion & wider plasma physics
  - Different configurations: common physics and technology questions



- Wide range of fundamental plasm physics with DP
- Organic molec in meteor tails

Tested SPARC error fiera correction & the rechnology

#### Incredible flexibility to answer key questions

51

(MHz)

0.0

-0.5 E



# DIII-D Provides Distinctive Capabilities Internationally and Basis to Lever US Collaborations



#### Develop techniques at high power density

- Flexibility to resolve & integrated innovative exhaust, core and wall solutions
  - High opacity, low v\*, high performance, burning plasma relevant conditions
- Physics basis to project

#### Long pulses test evolution & wall



 Material & PFC evolution

> Long pulse control

> > NSTX-U

### Larger devices test scaling

- JT-60SA)
- Projection
   to reactor
  - Operational techniques

#### Key physics & novel techniques

- Aspect ratio & Shape
- Extreme divertor geometry
- + Super Alfvénic ions & high  $\beta$

ASDEX Upgrade · Liquid metals

### Important to focus on insights that lever international progress





# **DIII-D Providing Critical Roles in Preparation for ITER**

#### **Distinctive Contributions:**

- Develop & accelerate early phase ITER research plan
   H mode access, ELM control, EC ramp up, DMS tests
- Resolution of transients & development of ITER control
  - Stability, ELMs, disruptions, runaway electrons
  - Control development & burn simulation
- Validated physics models to project and interpret behavior in relevant low rotation conditions in ITER
  - Turbulent transport in coupled, opaque, low  $\nu^{*}$  &  $\Omega$  regimes
  - Wall to core Tungsten transport, MHD turbulence and AEs
- Development of robust, controlled scenarios to reach or exceed Q=10 & determine path to Q=5 steady state

#### **Key Capabilities:**

Carbon → Tungsten wall

EC: torque-free electron heating with precise deposition control

**Balanced torque NBI** 

New quench systems

Integrated control

Pellets for high density

**3D coils** 

Advanced measurements

– Baseline, reduced current & high  $\beta$  paths with radiative solutions



US success and benefit in ITER – key for wider fusion path – <u>requires</u> DIII-D to prepare techniques, tools & team

# DIII-D Provides a Critical and Cost-Effective Tool to Make Rapid Progress on the Bold Decadal Vision



- Highly flexible user facility able to pioneer the tokamak path to FPP
  - -Tokamak serves as 'first integrator' to resolve fusion technologies more broadly
- Critical enabler of the wider vision & the private sector through technology testing, its flexibility & measurement systems, and sharing its expertise
- Serves entire fusion community well & productive now on all these goals
  - -Developing people, sharing knowhow, enabling success in private sector & ITER



Get in touch if you want to participate!

RJ Buttery/Open Call Background 2025/26